lmeca2648

2017

UCLouvain

Nuclear thermal-hydraulics.

6 credits	40.0 h + 7.5 h	Q1

Teacher(s)	Bartosiewicz Yann ;
Language :	English
Place of the course	Louvain-la-Neuve
Main themes	• Reactor heat generation • Transport equations (single-phase & two-phase flow) • Thermal analysis of fuel elements • (Single-phase fluid mechanics and heat transfer)'usually already known • Two-phase flow dynamics • Two-phase heat transfer • Single heated channel; steady state analysis • Single heated channel; transient analysis • Flow loops • Utilisation of established codes and introduction to advanced topics (modelling and thermalhydraulics for GEN4 reactors)
Aims	1 • To be familiarised with various reactor types and their main design and operational characteristics • To learn how to estimate the volumetric heat generation rate in fission reactor cores under normal operation and shutdown conditions • To learn how to analyse the thermal performance of nuclear fuel elements • To learn the basic fluid mechanics of single phase reactor cooling systems • To learn to calculate pressure drop in reactor systems, including tube bundles, and spacer grids • To learn to analyse the heat transfer characteristics of single phase reactor cooling systems • To learn the basic fluid mechanics of two-phase systems, including flow regime maps, void-quality relations, pressure drop, and critical flow • To learn the fundamentals of boiling heat transfer, and its implications for reactor design • To learn the fundamentals of core thermal design, with attention to design uncertainty analysis and hot channel factors. - - - - *The contribution of this Teaching Unit to the development and command of the skills and learning outcomes of the programme(s) can be accessed at the end of this sheet, in the section entitled "Programmes/courses offering this Teaching Unit".*
Evaluation methods	Closed book - oral
Teaching methods	• 2 t.m.: 40h teaching + seminar and 15h practical works in classroom • SCK.CEN guidance for demonstrations with codes • SCK.CEN + UCL TA for practical works The course takes place at the Nuclear Research Centre of Belgium (SCK.CEN) in gthe framework of the BNEN interuniversity programme (see: http://www3.sckcen.be/bnen/). One makes use of the software available at the research centre. Courses taking place at SCK.CEN are condensed over a period of 1 to 3 intensive weeks of courses, according to the number of ECTS.
Content	• Reactor heat generation • Transport equations (single-phase & two-phase flow) • Thermal analysis of fuel elements • (Single-phase fluid mechanics and heat transfer)'usually already known • Two-phase flow dynamics • Two-phase heat transfer • Single heated channel; steady state analysis • Single heated channel; transient analysis • Flow loops • Utilisation of established codes and introduction to advanced topics (modelling and thermalhydraulics for GEN4 reactors)
Inline resources	http://icampus.uclouvain.be/claroline/course/index.php?cid=MECA2648

Bibliography	**REFERENCE BOOKS ON THE CONTENT**
	• Todreas, N.E. and Kazimi, M.S. Nuclear System I: Thermal Hydraulic Fundamentals, Hemisphere Publishing Corp., New York, 1990 • Todreas, N. E. and Kazimi, M.S. Nuclear Systems II: Elements of Thermal Hydraulic Design, Hemisphere Publishing Corp., New York, 1990.
Faculty or entity in charge	MECA

Programmes containing this learning unit (UE)				
Program title	Acronym	Credits	Prerequisite	Aims
Master [120] in Electro-mechanical Engineering	ELME2M	6		🔍
Master [120] in Mechanical Engineering	MECA2M	6		🔍