

LMECA2648 2013-2014

Nuclear thermal-hydraulics.

6.0 credits

40.0 h + 7.5 h

2q

Teacher(s) :	Bartosiewicz Yann ;
Language :	Anglais
Place of the course	Louvain-la-Neuve
Inline resources:	> http://icampus.uclouvain.be/claroline/course/index.php?cid=MECA2648
Main themes :	Reactor heat generation Transport equations (single-phase & mp; two-phase flow) Thermal analysis of fuel elements (Single-phase fluid mechanics and heat transfer)'usually already known Two-phase flow dynamics Two-phase heat transfer Single heated channel; steady state analysis Single heated channel; transient analysis Flow loops Utilisation of established codes and introduction to advanced topics (modelling and thermalhydraulics for GEN4 reactors)
Aims :	 To be familiarised with various reactor types and their main design and operational characteristics To learn how to estimate the volumetric heat generation rate in fission reactor cores under normal operation and shutdown conditions To learn how to analyse the thermal performance of nuclear fuel elements To learn the basic fluid mechanics of single phase reactor cooling systems To learn to calculate pressure drop in reactor systems, including tube bundles, and spacer grids To learn to calculate pressure drop in reactor systems, including flow regime maps, void-quality relations, pressure drop, and critical flow To learn the fundamentals of boiling heat transfer, and its implications for reactor design To learn the fundamentals of core thermal design, with attention to design uncertainty analysis and hot channel factors. The contribution of this Teaching Unit to the development and command of the skills and learning outcomes of the programme(s) can be accessed at the end of this sheet, in the section entitled "Programmes/courses offering this Teaching Unit".
Evaluation methods :	Closed book - oral

Teaching methods :	2 t.m.: 40h teaching + seminar and 15h practical works in classroom SCK.CEN guidance for demonstrations with codes SCK.CEN guidance for demonstrations with codes SCK.CEN + UCL TA for practical works The course takes place at the Nuclear Research Centre of Belgium (SCK.CEN) in gthe framework of the BNEN interuniversity programme (see: http://www3.sckcen.be/bnen/). One makes use of the software available at the research centre. Courses taking place at SCK.CEN are condensed over a period of 1 to 3 intensive weeks of courses, according to the number of ECTS.
Content :	Reactor heat generation Transport equations (single-phase & mp; two-phase flow) Thermal analysis of fuel elements (Single-phase fluid mechanics and heat transfer)'usually already known Two-phase flow dynamics Two-phase heat transfer Single heated channel; steady state analysis Single heated channel; transient analysis Flow loops Utilisation of established codes and introduction to advanced topics (modelling and thermalhydraulics for GEN4 reactors)
Bibliography :	REFERENCE BOOKS ON THE CONTENT Todreas, N.E. and Kazimi, M.S. Nuclear System I: Thermal Hydraulic Fundamentals, Hemisphere Publishing Corp., New York, 1990 Todreas, N. E. and Kazimi, M.S. Nuclear Systems II: Elements of Thermal Hydraulic Design, Hemisphere Publishing Corp., New York, 1990.
Cycle and year of study :	 Master [120] in Mechanical Engineering Master [120] in Electro-mechanical Engineering
Faculty or entity in charge:	MECA